



Wir schaffen Wissen – heute für morgen

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Use of SERPENT at PSI/EPFL for thermal system analysis.



Activities Involving SERPENT at PSI/EFPL

BWR analysis with SERPENT/SIMULATE-3 code sequence

- **Motivations**
- **Methodology : Two-Step Approach**
- **Results**
 - Cross Sections
 - Full core Comparison

Modeling of CROCUS with SERPENT/PARCS code sequence

- **Motivations**
- **Methodology**
- **Results**

Conclusion and Outlook

For fast system analysis

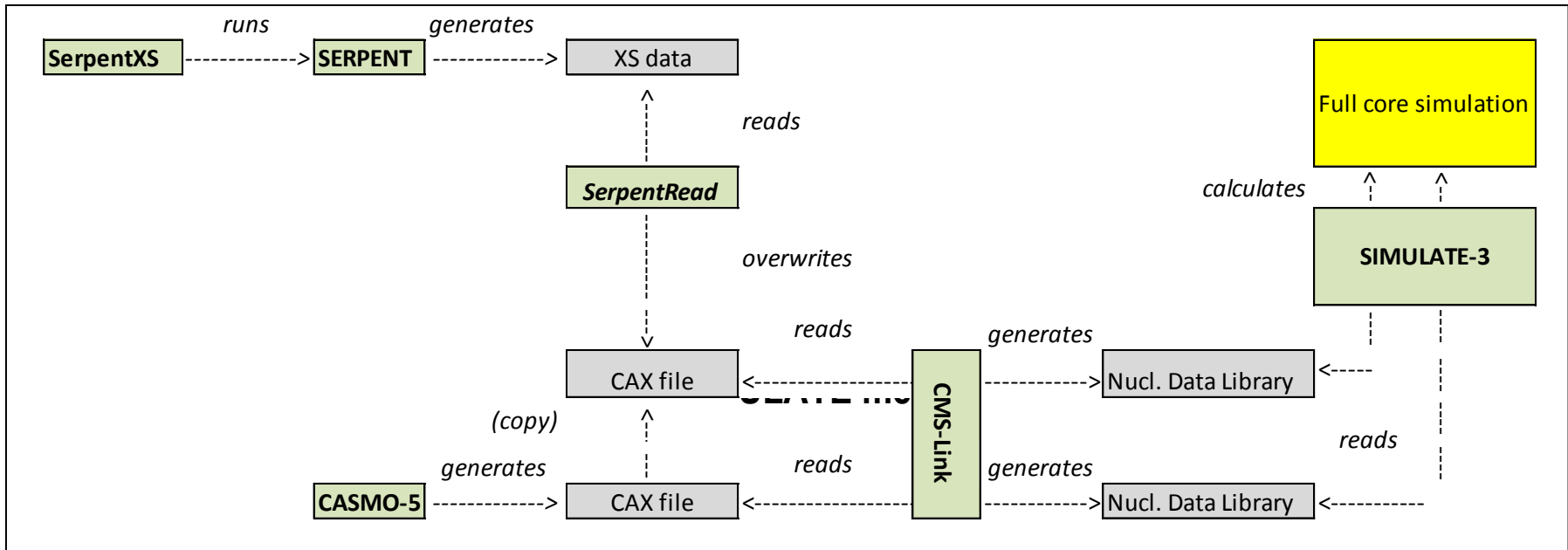
- See Sandro Pelloni's presentation

For thermal system analysis

- **Development of SERPENT/SIMULATE-3 scheme for BWR core analysis**
- Reflector modelling with SERPENT for PWR Core analysis (reduce bias near reflectors)
- Use of SERPENT for Burnup Credit
 - Validation of SERPENT for depletion (against P.I.E data)
 - Coupling of SERPENT to FALCON
- **Assessment of SERPENT for long term decay (geological repository)**

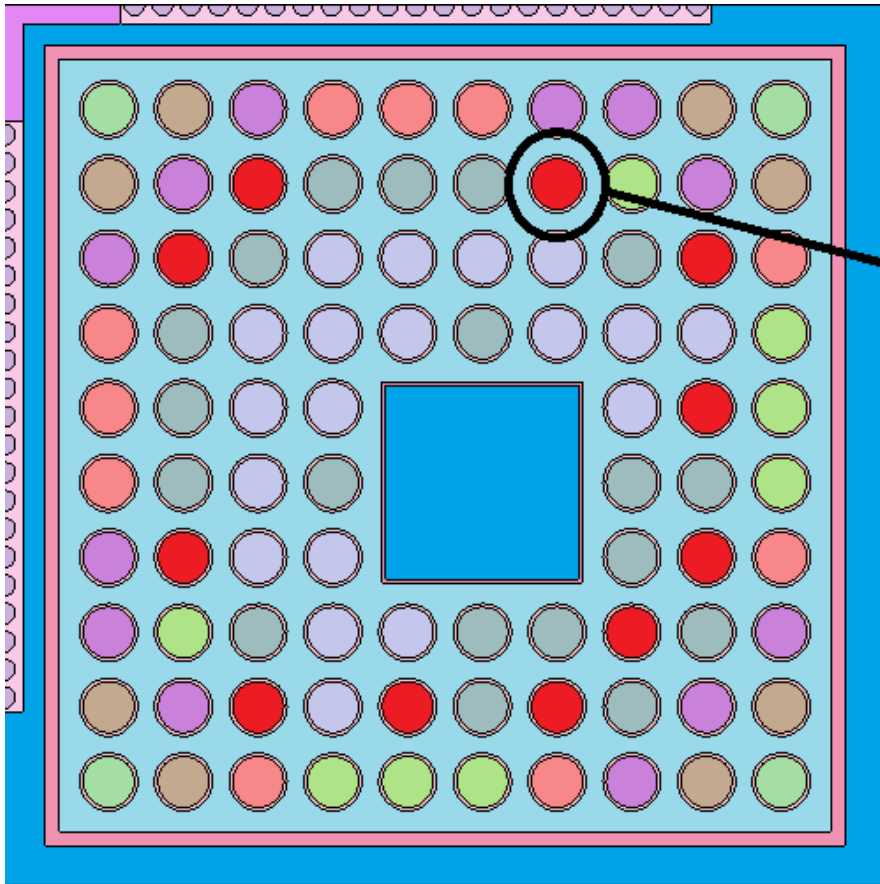
At Ecole Polytechnique Federale de Lausanne

- **Use of SERPENT/PARCS to model steady state and transient behavior CROCUS research reactor**
- Long term goal is to update Safety Report of CROCUS with state-of-the-art methods



Approach:

- Use **CASMO** output (Cax file) as vector to insert **SERPENT XS** into **SIMULATE-3**
- List of **CASMO** nuclear data being overwritten using **SERPENT** results:
 - 2 group constants and assembly discontinuity factors (D from B1 calculation)
 - Consistency with other information (Xe microscopic xs for example) may not be insured.
- First compare **XS** between **CASMO** and **SERPENT**, then **SIMULATE-3** calculations for a BWR full core



Pin with Gd

Based on ATRIUM-10 Specifications

SERPENT Models

- 10000x(500+20) n
- 6000x(100+20) n – “Low Neutron Count”

Stochastic Uncertainty

- *k-inf*: std \approx 30/100 pcm
- Cross sections: std \approx 0.03/0.1%

S3C default history/branch case matrix not used

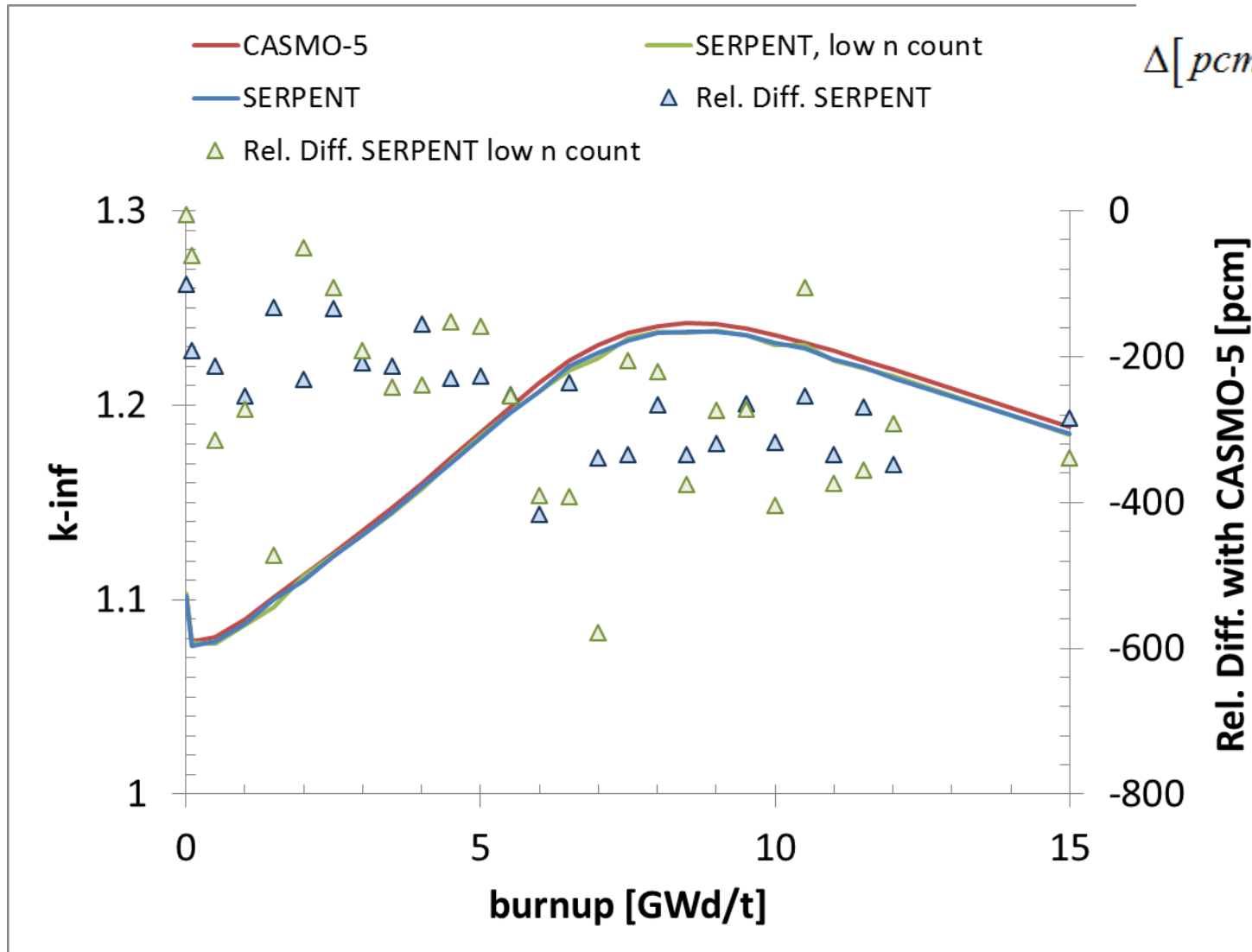
- Too computationally expensive
- Simplified to be acceptable for SIMULATE-3

CPU cost for 136 transport calculations

- 20min for CASMO-5
- 24h for SERPENT (4 processors)

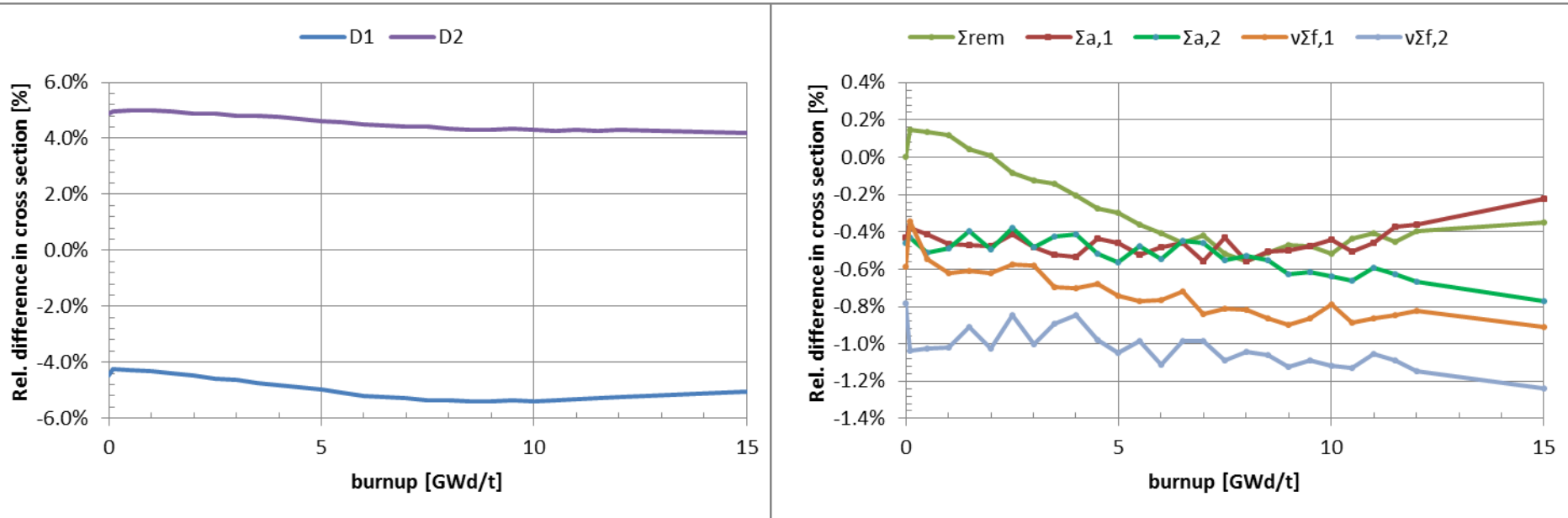
History	Branch		Transport Calculations
Reference	Depletion (Void 40%, fuel temperature 729 K, moderator temperature 559 K)		26
	Branches	Void 40%	5
		Void 80%	5
		Rodded	5
		Fuel temperature 559 K	5
		Fuel temperature 1500 K	5
		Coolant/moderator temp. 293 K, [Void 0]	5
Void 0	Depletion (Void 0%, fuel temperature 729 K, moderator temperature 559 K)		24
	Branches	Void 40%	5
		Void 80%	5
		Rodded	5
		Fuel temperature 559 K	5
		Fuel temperature 1500 K	5
		Coolant/moderator temp. 293 K, [Void 0]	5
Rodded	Reference (Void 40%, fuel temperature 729 K, moderator temperature 559 K, rodDED)		32
	Branches	Control rod withdrawn	5

k-inf comparison for Reference History



$$\Delta[pcm] = \frac{Q_{SERP} - Q_{C5}}{Q_{C5}} \cdot 10^5$$

$$Q = k_{inf}$$



Good agreement besides the diffusion coefficients – known issue

No new error pattern with exposure

- Slight decrease of difference with burnup for removal xs
- Slight increase of difference with burnup for absorption

$$\Delta[\%] = \frac{Q_{SERP} - Q_{C5}}{Q_{C5}} \cdot 100$$

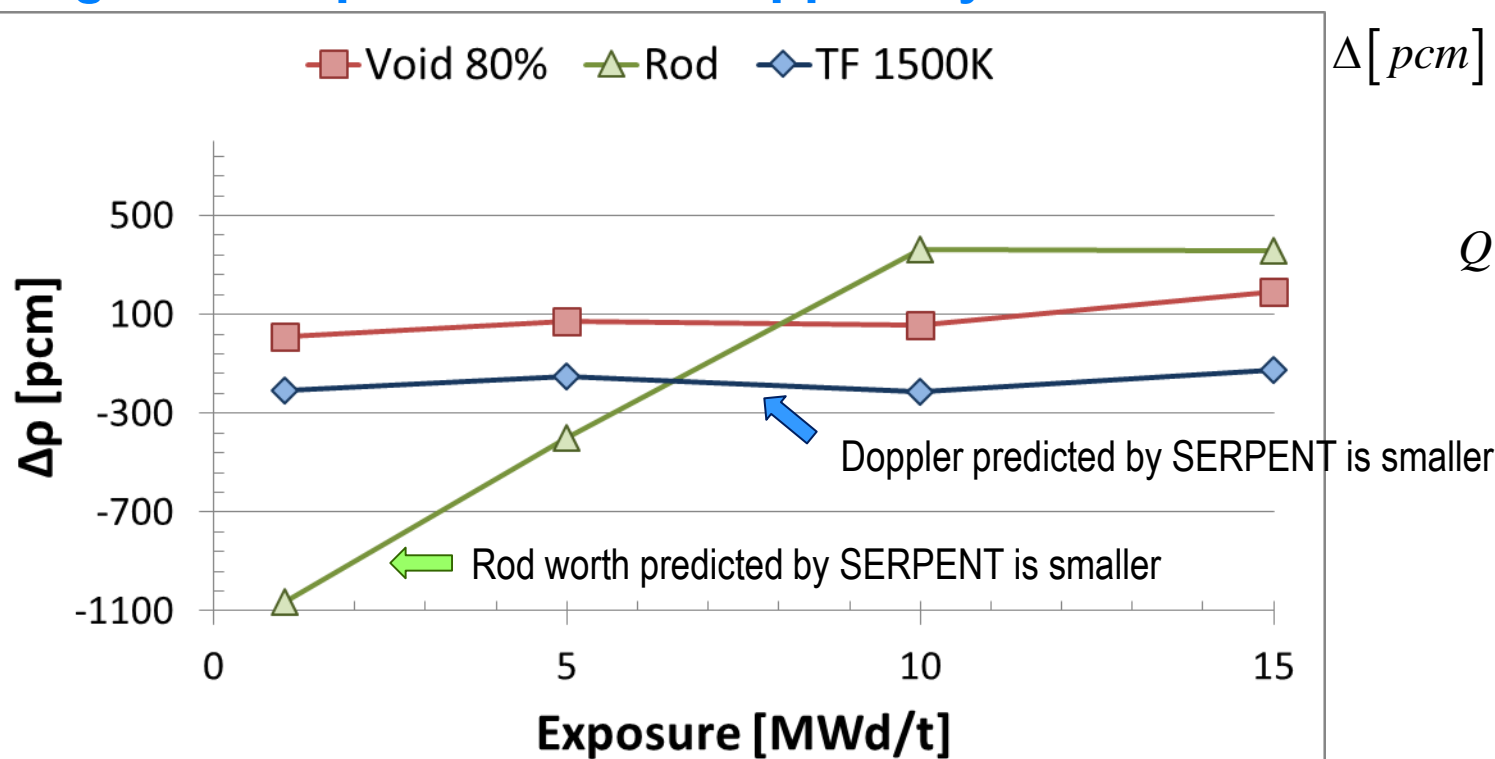
$$Q = \Sigma$$

Reactivity differences between CASMO-5 and SERPENT due to perturbation of instantaneous variables – branch calculations

- 80% void, Insertion of control rod, 1500K Fuel Temperature

Control Rod Insertion is not captured well

Slight underprediction of Doppler by SERPENT



$$\Delta[pcm] = (Q_{SERP} - Q_{C5}) \cdot 10^5$$

$$Q = \frac{1}{k_{inf}^{pert}} - \frac{1}{k_{inf}^{ref}}$$

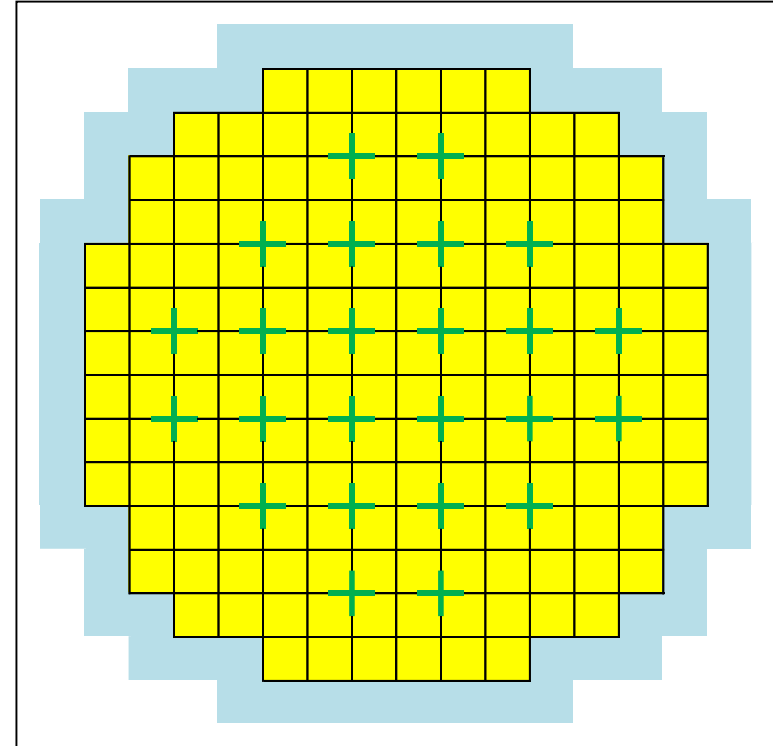
BWR Core Model

- 224 fuel assemblies
- All identical (!), only one segment
- Reflector XS are generated by CASMO-5

Model leads to unrealistic power shape

Comparison of CASMO-5/SIMULATE-3 and SERPENT/SIMULATE-3

- At Hot Zero Power (no control rods)
- During a reactor cycle (8.5 MWd/T)

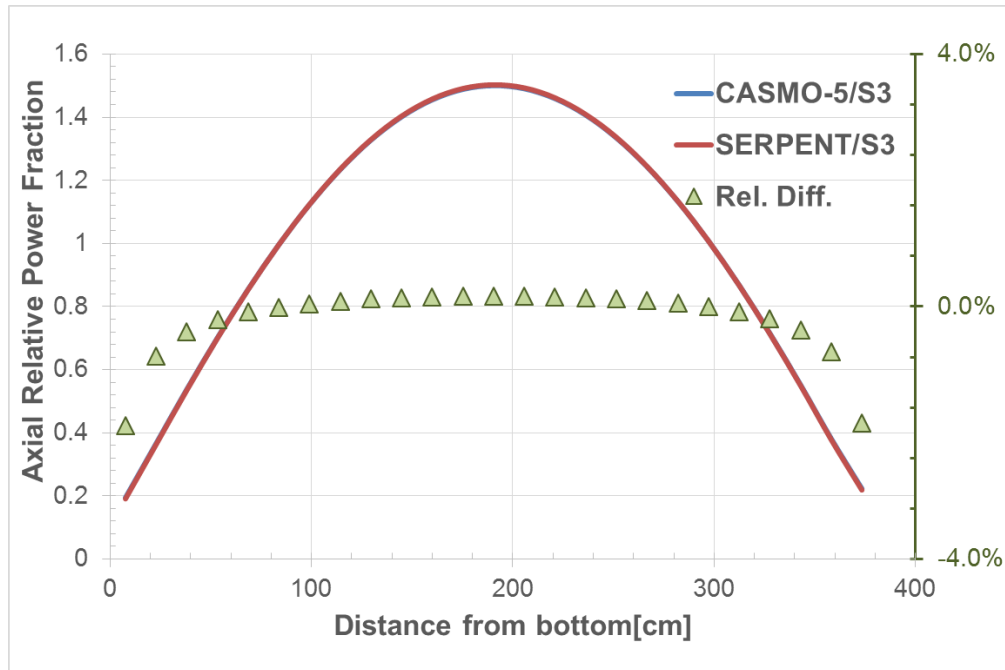


Case		k-eff C5/S3	$\Delta k/k$ [pcm]	$\Delta k/k$ [pcm] Low n count model
HZP		1.10561	-148	82
HFP		1.03527	-112	224
Cycle Calculation				
Exposure	0.1	1.03470	-228	-62
	1	1.06164	-94	-7
	2	1.11522	-84	-39
	4	1.13155	-419	-218
	6.5	1.12566	-488	-319
	8.5	1.12343	-459	-277

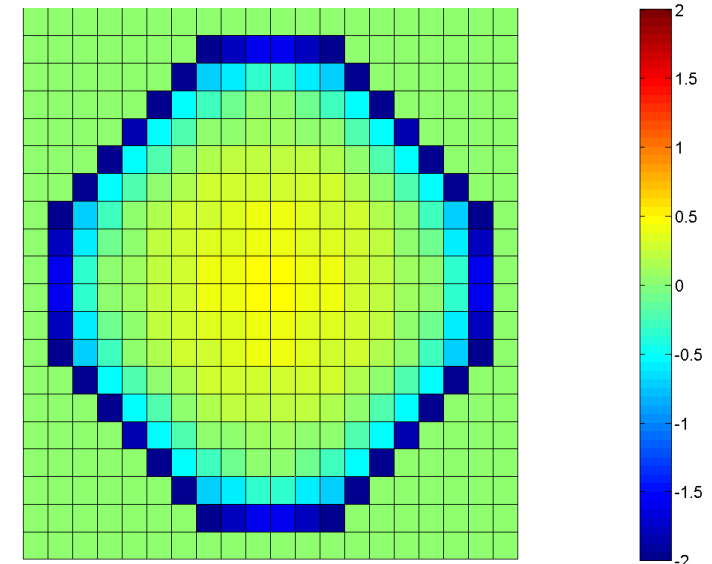
Conclusion

- Acceptable agreement at HZP and HFP
- Increase of discrepancies with exposure
- Large increase of discrepancies at 0.1 GWd/T due to inconsistency in Xe/Sm information
- Effect of stochastic uncertainty is large (~200pcm)

Axial Relative Power Fraction



Radial Relative Power Fraction



Relative Difference Expressed As:

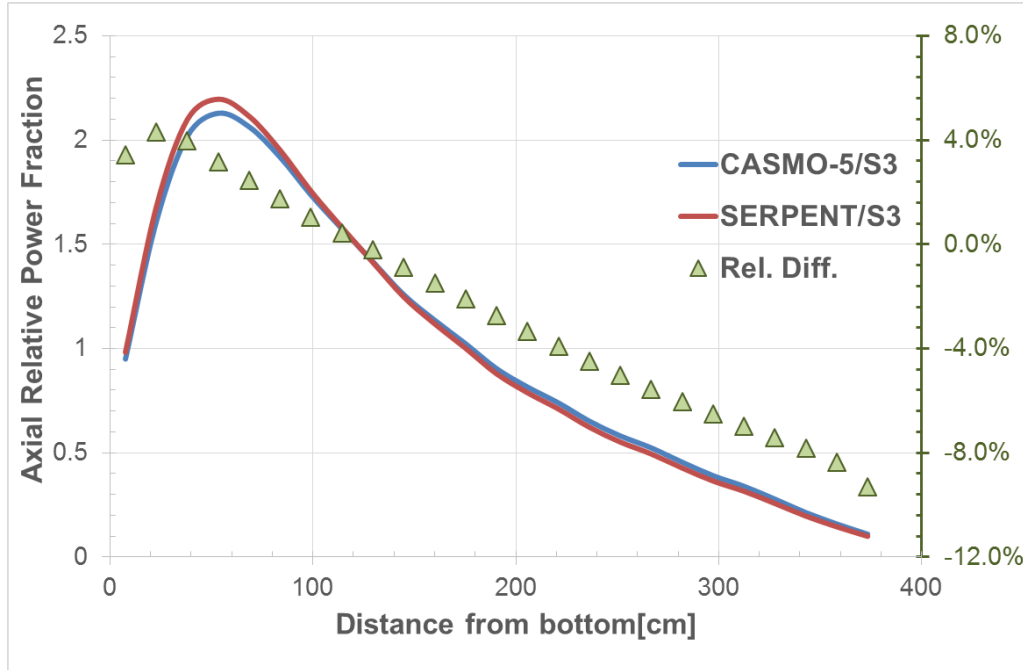
$$\Delta[\%] = \frac{Q_{SERP/S3} - Q_{C5/S3}}{Q_{C5/S3}} \cdot 100$$

Overall good agreement

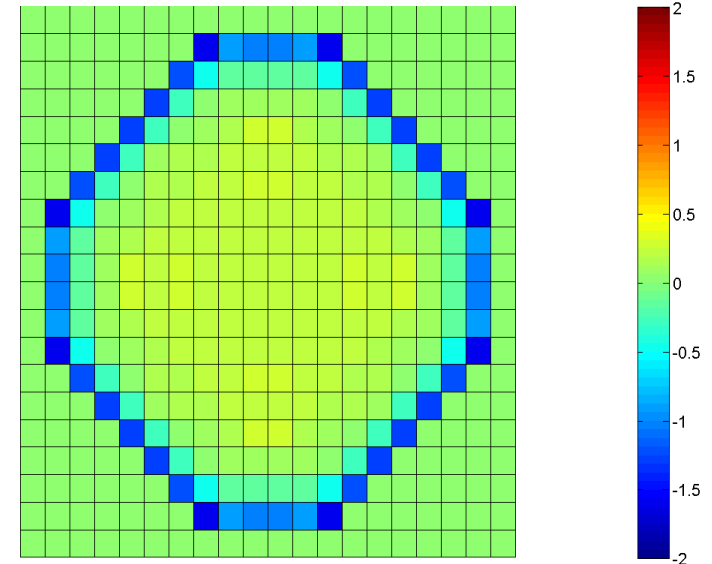
- Difference in diffusion coefficients are seen near the reflectors
- SERPENT/SIMULATE-3 overpredicts leakage

$$Q = RPF$$

Axial Relative Power Fraction



Radial Relative Power Fraction



Relative Difference Expressed As:

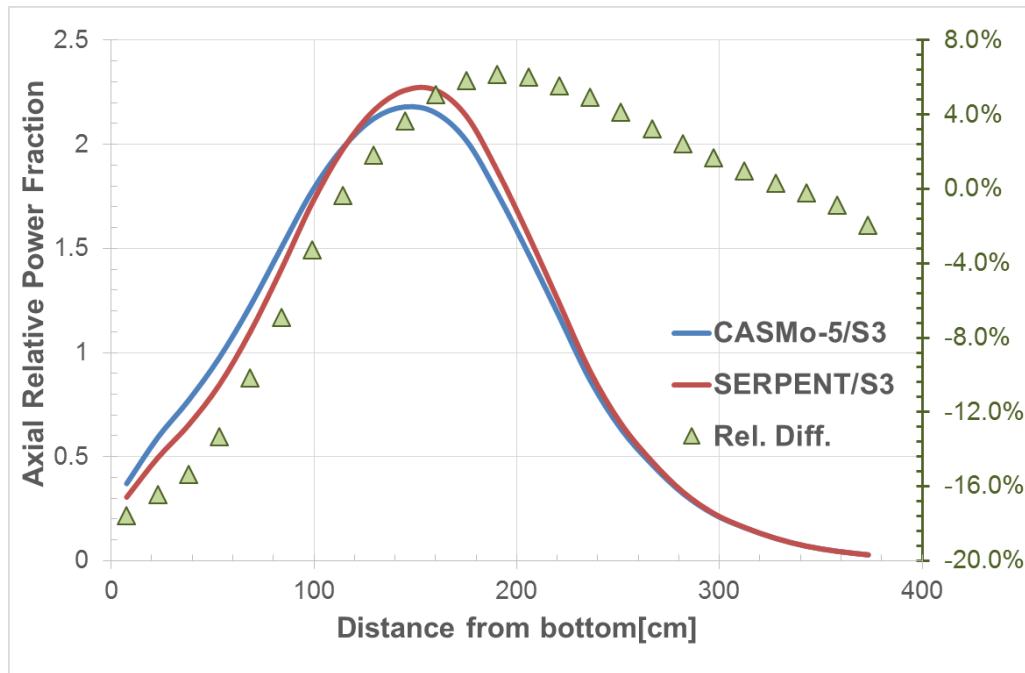
$$\Delta[\%] = \frac{Q_{\text{SERP/S3}} - Q_{\text{C5/S3}}}{Q_{\text{C5/S3}}} \cdot 100$$

Overall good agreement

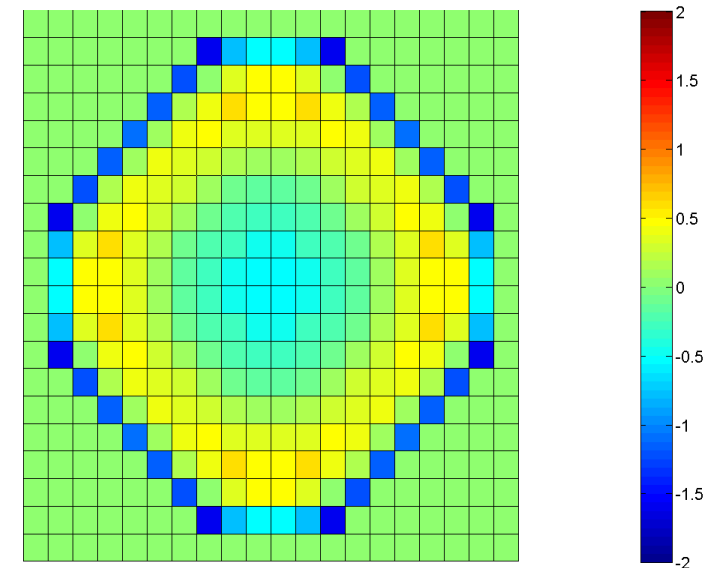
- Overprediction of the power peaking by SERPENT/SIMULATE-3
- Difference due to handling of instantaneous variables only
- Consistent with cross section comparison
- SERPENT/SIMULATE-3 overpredicts leakage

$$Q = RPF$$

Axial Relative Power Fraction



Radial Relative Power Fraction



Relative Difference Expressed As:

$$\Delta[\%] = \frac{Q_{\text{SERP/S3}} - Q_{\text{C5/S3}}}{Q_{\text{C5/S3}}} \cdot 100$$

$$Q = RPF$$

Agreement degrades with exposure

- Differences due to branch and history effects
- Radial RPF: underprediction of power by SERPENT/SIMULATE-3 at high exposure

Good agreement between CASMO-5 and SERPENT for BWR XS generation

- Except for diffusion coefficients
- Similar behavior with exposure
- Cost of SERPENT calculations is large and may be prohibitive for fuel segments

Comparison of BWR full core calculation with SIMULATE-3

- Acceptable agreement at HZP and HFP
- Increase of discrepancies with exposure
- Large increase of discrepancies at 0.1 GWd/T due to inconsistency in Xe/Sm information
- Effect of stochastic uncertainty is surprisingly large

CROCUS is a zero power (100W) educational and research reactor

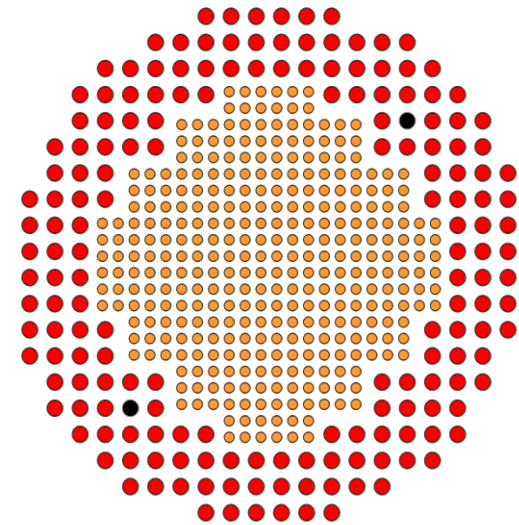
- Mix of UO₂ and Umetal fuels
- H₂O moderated
- Started operation in 1983 (EPFL)

Current Modelling

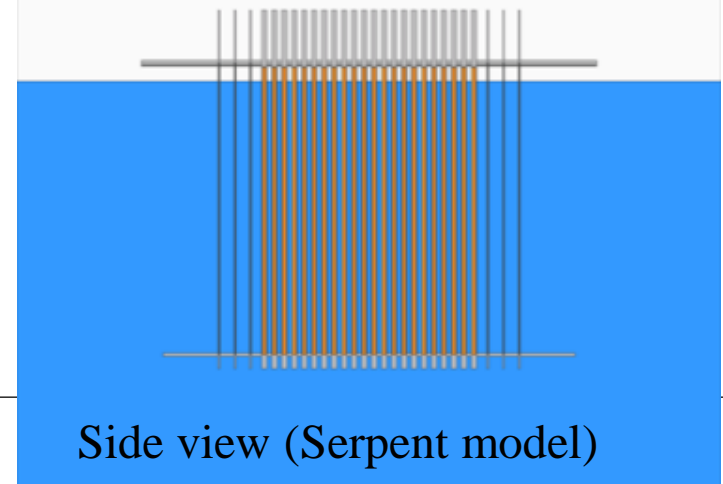
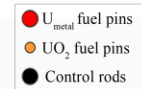
- 4 zones, 2G diffusion
- Point kinetics
- Simplified Thermal Hydraulic

Goal is to improve current modelling capabilities

- TRACE/PARCS/SERPENT code scheme
- Today focus on PARCS/SERPENT



Top view

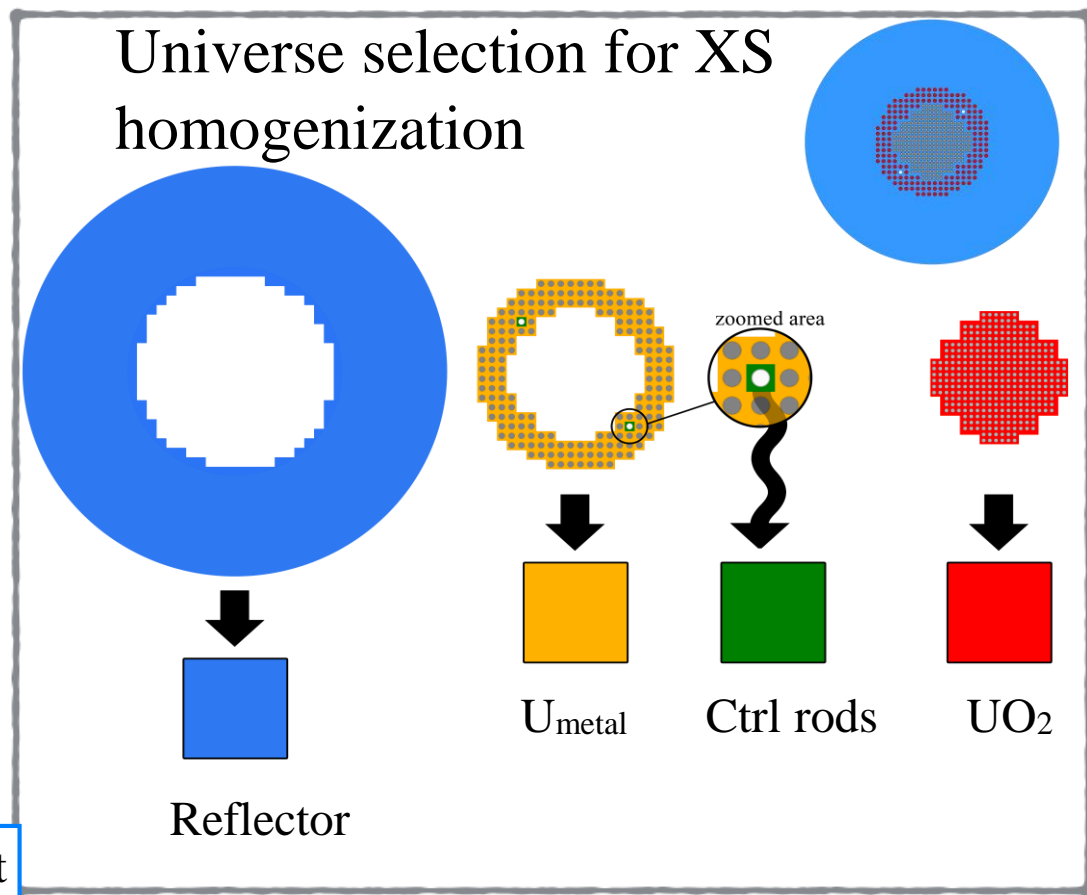
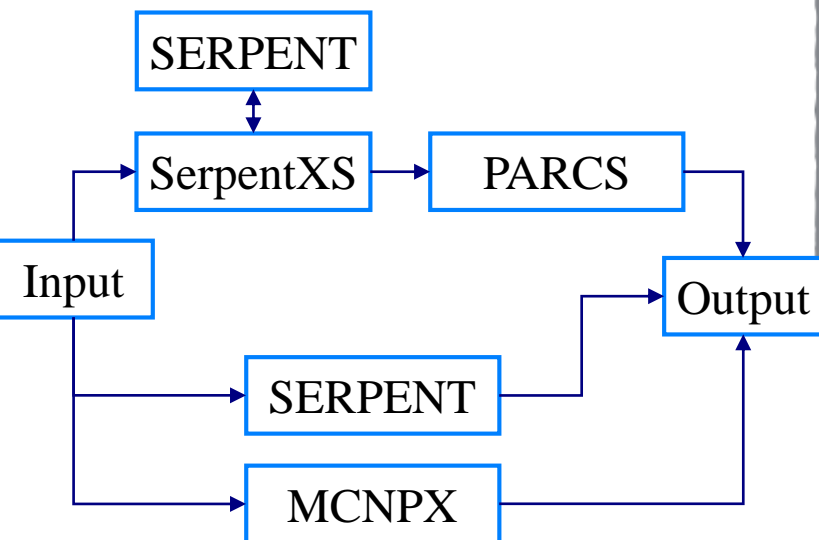


SERPENT v1.1.19 for XS generation

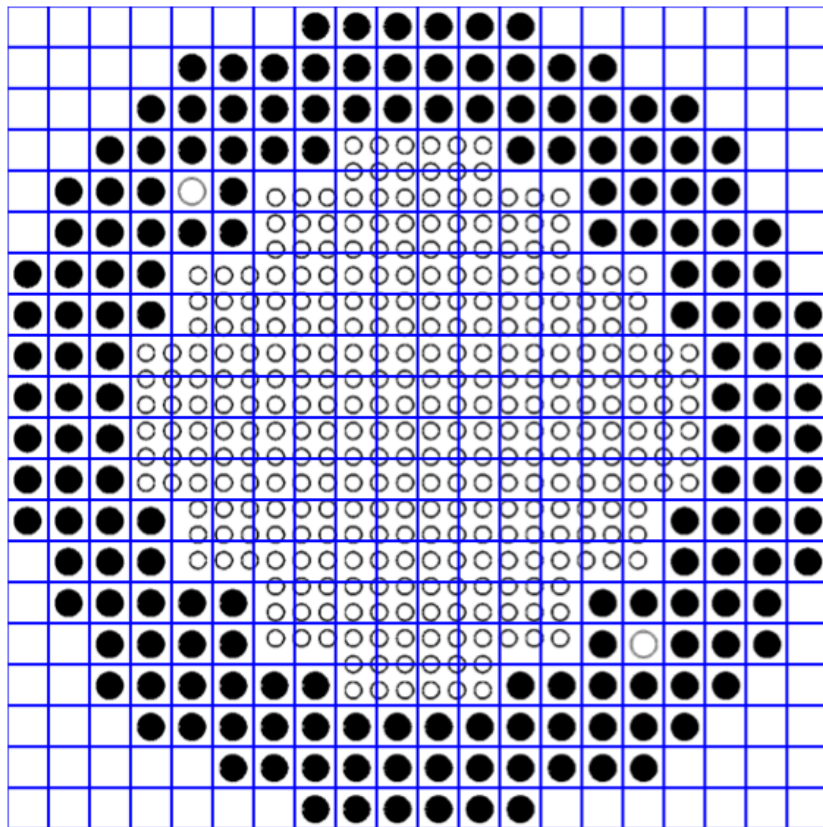
SerpentXS for XS formatting

Methodology

- Using full core solution for XS generation
- 2G XS

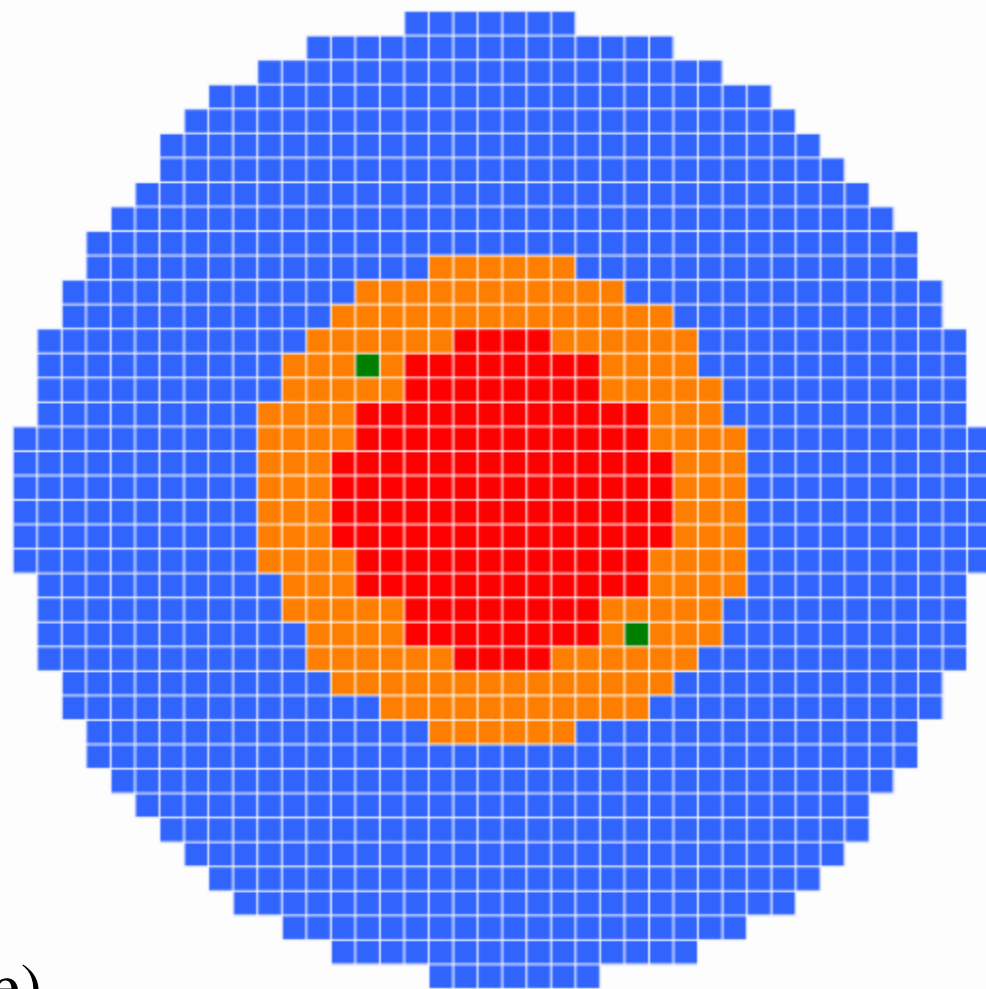


U_{metal} cell size for PARCS nodes



PARCS nodes \Leftrightarrow UO_2 fuel pins
becomes complex (see inner lattice)

PARCS model



Four set of cross sections

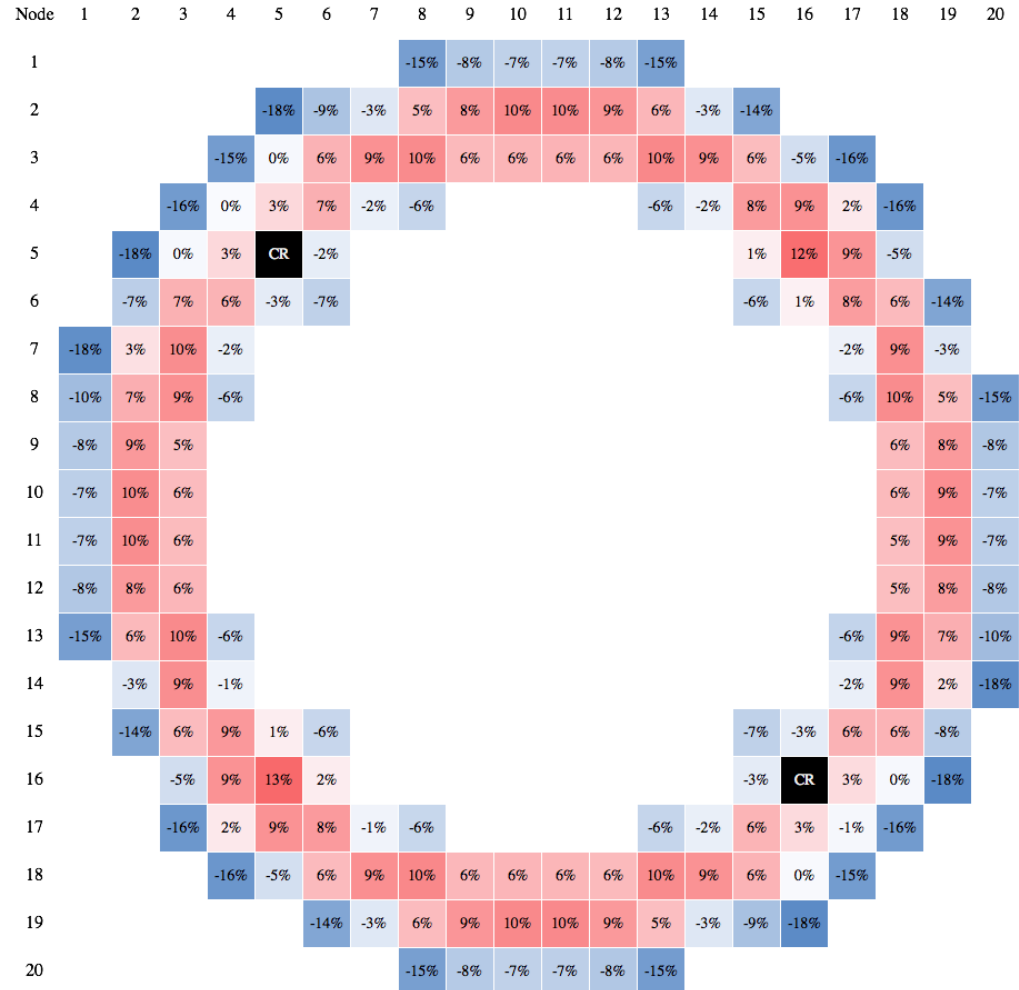
Eigenvalue comparison

Model	keff	Rel. Diff. [pcm]
SERPENT	$1.00189 \pm 5 \text{ pcm}$	-
MCNP5	$1.00202 \pm 5 \text{ pcm}$	13
PARCS	0.99793	-385

Ctrl. rod worth comparison

Model	Reactivity Worth ($\Delta\rho$)	Rel. Diff. [pcm]
Serpent	$169 \pm 6 \text{ pcm}$	-
MCNP5	$174 \pm 7 \text{ pcm}$	5 ± 9
PARCS	136 pcm	-33

Relative power fraction % difference
 $(RPF_{\text{PARCS}} - RPF_{\text{Serpent}}) / RPF_{\text{Serpent}}$



Development of a SERPENT/PARCS code sequence for the modeling of the CROCUS reactor

- Development of an adequate homogenization scheme thanks to SERPENT
- Development of an approximate core model with PARCS

Comparison to direct SERPENT solution is not completely satisfactory

- Eigenvalue is within 400pcm
- Control rod worth is within 30pcm
- Large discrepancies in power prediction

Continue Assessment of SERPENT/SIMULATE-3 code sequence for BWR analysis

- Real core model
- Overwrite more CASMO information with SERPENT data
- Move to SERPENT 2

Improvement of SERPENT/PARCS CROCUS model

- Consider a more detailed energy group structure
- Use B1 definition for the diffusion coefficient
- Validates model against experimental measurements
- Extend to transient analysis

Thank you for your attention, Questions?

