

Investigating Variable Fidelity Monte Carlo with Serpent Fixed Source Mode

7th International Serpent User Group Meeting
Gainesville, Florida, USA
November 6-9, 2018

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DATE: 7.11.2017

Engineering - Energy, Fluid dynamics and Turbo-machinery

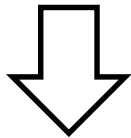
Presentation Outline

1. What is “Variable Fidelity” Monte Carlo?
2. MONK 10A and WIMS
3. Methodology
4. Accuracy of simple approach
5. Scope for improvement
6. Conclusions

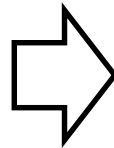
Variable Fidelity Monte Carlo

Approximations to Monte Carlo:

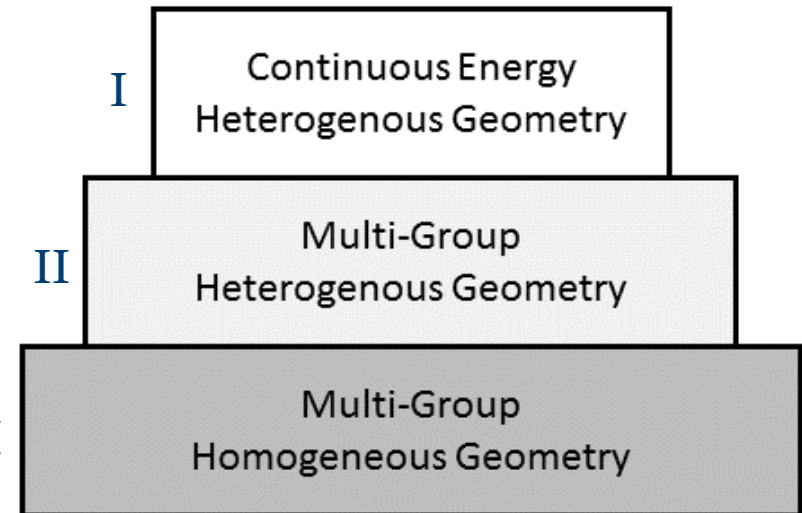
- In Angle
- In Energy (MG Data)
- In Space (Homogenisation)



- Approximation in angle is usually determined by Nuclear Data
- Homogenisation and CE Data is not practical



Monte Carlo Fidelity Levels



Calculation in 2nd level: ~ 5-6 Speed-up [KENO]^[1]

Calculation in 3rd level: ~ 5-7 Speed-up [MORA]^[2]

[1] Goluoglu, Sedat, Lester M. Petrie, Michael E. Dunn, Daniel F. Hollenbach, and Bradley T. Rearden. "Monte Carlo Criticality Methods and Analysis Capabilities in SCALE." Nuclear Technology, 2011.

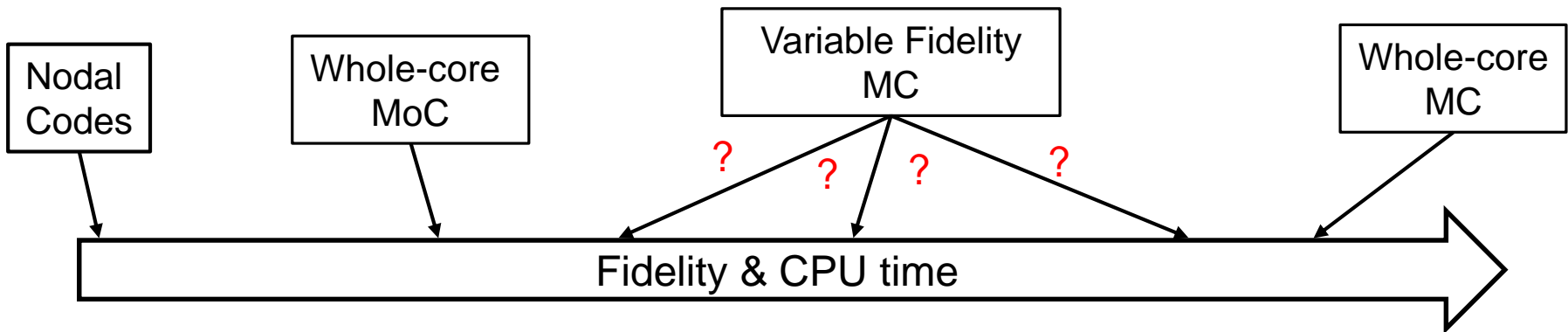
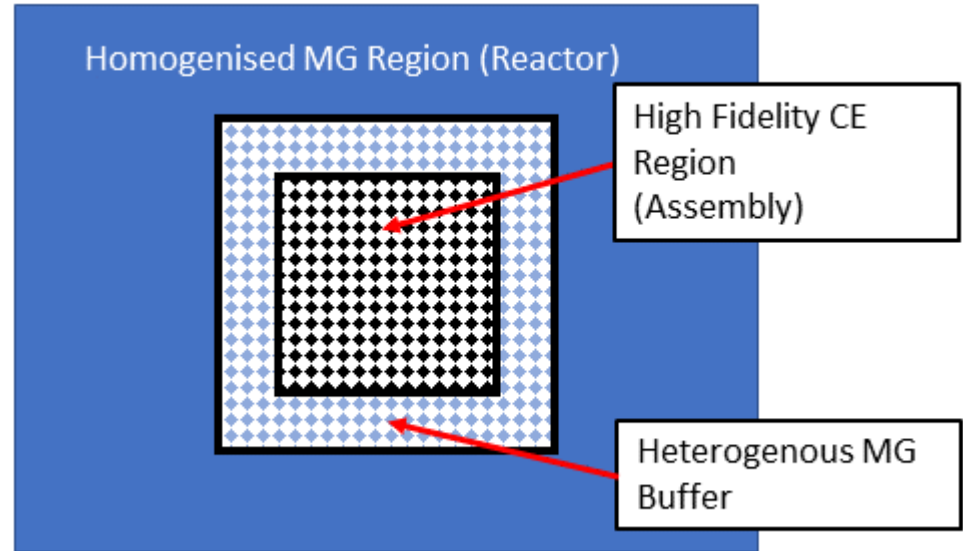
[2] Leppänen, Jaakko. "On the Feasibility of a Homogenised Multi-Group Monte Carlo Method in Reactor Analysis." Casino-Kursaal Conference Center, Interlaken, Switzerland, 2008.

Variable Fidelity Monte Carlo

Question:

Is it possible to accelerate MC solution by using lower fidelity in regions where accuracy is less important?

i.e. Temperature margin calculation
→ accuracy matters for pin cells close to a limit



MONK 10A and WIMS

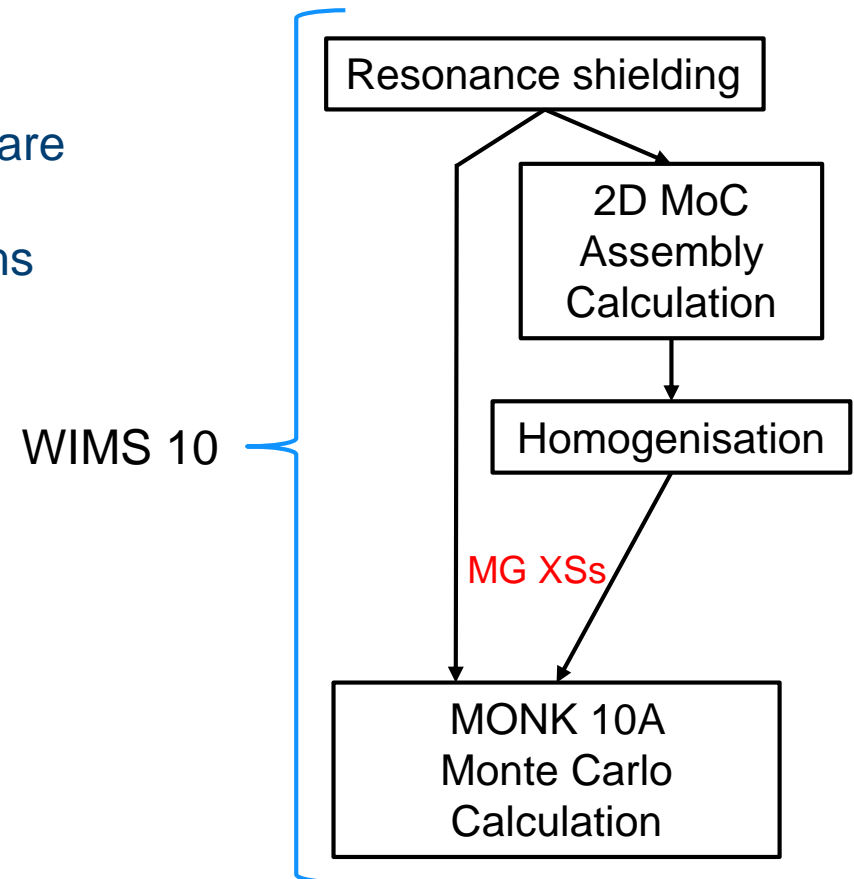
Serpent does not support MG data → Another code was needed

MONK 10A:

- MC code developed by ANSWERS Software
- Designed for criticality calculations
- Some support for reactor physics problems (more in 10B)
- Support of MG data
- Integrated into WIMS 10

WIMS 10:

- Lattice physics code suite
- MoC and Collision Probability
- Equivalence and Sub-Group Resonance treatment



Approximating “Variable Fidelity”

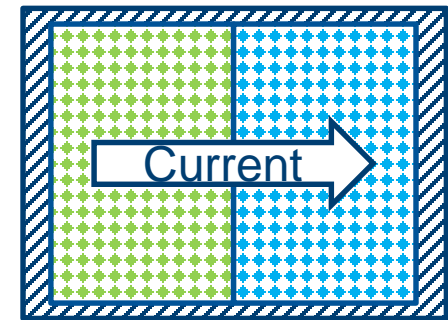
Methodology:

1. Perform low fidelity eigenvalue calculation
2. Sample Current crossing the boundary
3. Replace part of the geometry with current source and vacuum BC
4. Run Fixed Source Calculation

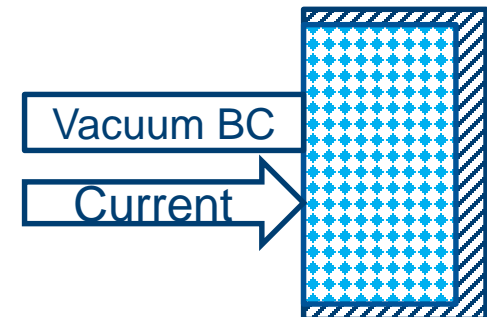
Limitations:

1. Only systems with $k=1$
2. Only current spectrum was sampled

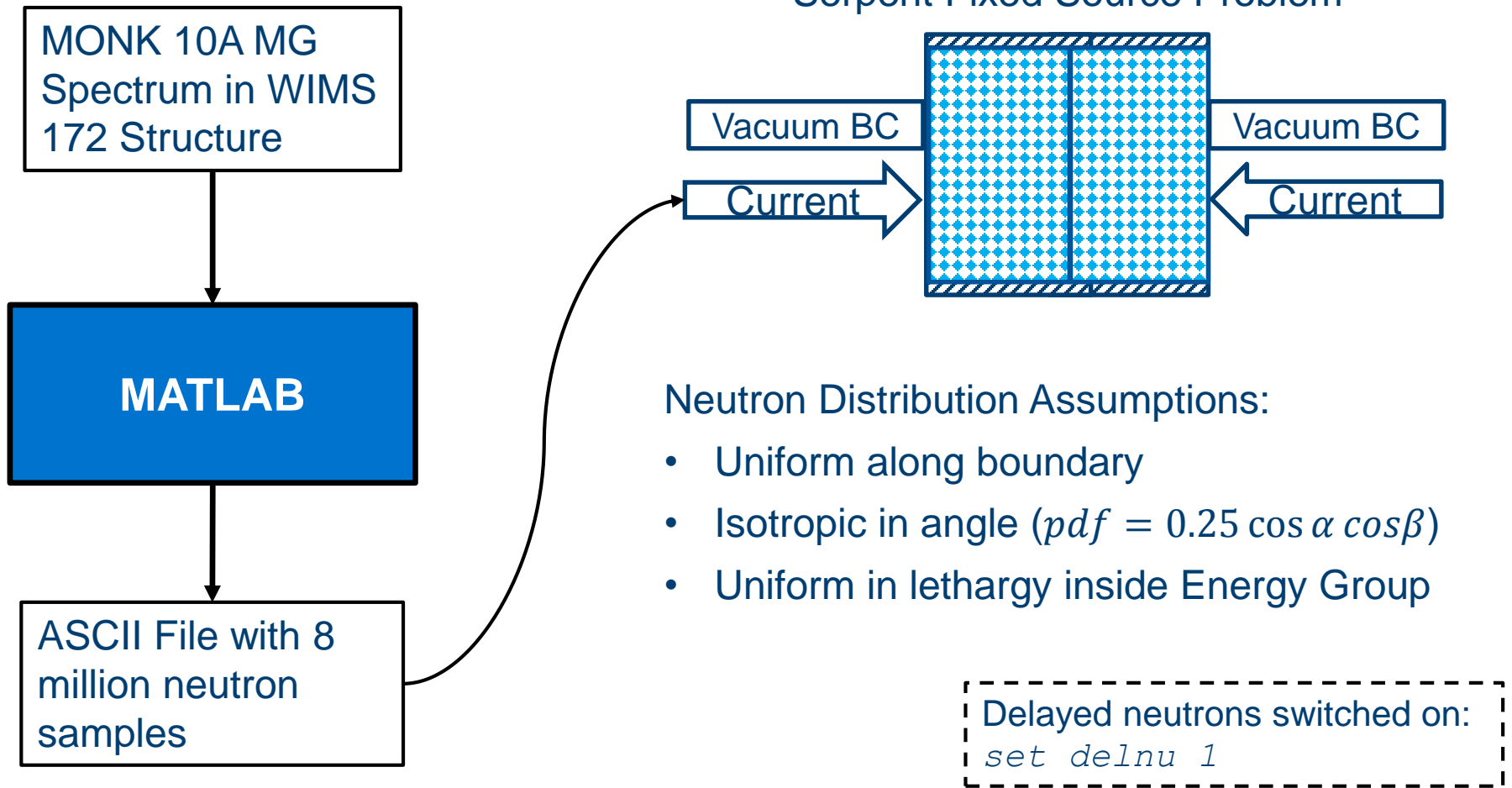
MG Problem (MONK 10A)



CE Problem (Serpent)

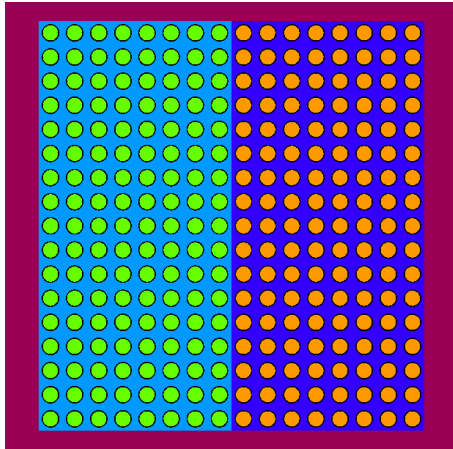


Serpent Fixed Source Calculation



Test Case

Geometry Schematic



Row-by-Row Fuel Composition

| | | | | | | | | | | | | | | | |
|---|------------|---------------------|------------|---------------------|------------|---------------------|------------|------------|------------|-------------|------------|---------------------|------------|-------------|---------------------|
| 1 | 2 | 3 | 4 | 5 | 6 | 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 |
| | 4.3wt% MOX | 2.5wt% Enriched UOX | 4.3wt% MOX | 2.5wt% Enriched UOX | 4.3wt% MOX | 2.5wt% Enriched UOX | 4.3wt% MOX | 4.3wt% MOX | 8.7wt% MOX | 2wt% Gd UOX | 8.7wt% MOX | 2.5wt% Enriched UOX | 4.3wt% MOX | 2wt% Gd UOX | 2.5wt% Enriched UOX |

Left Assembly

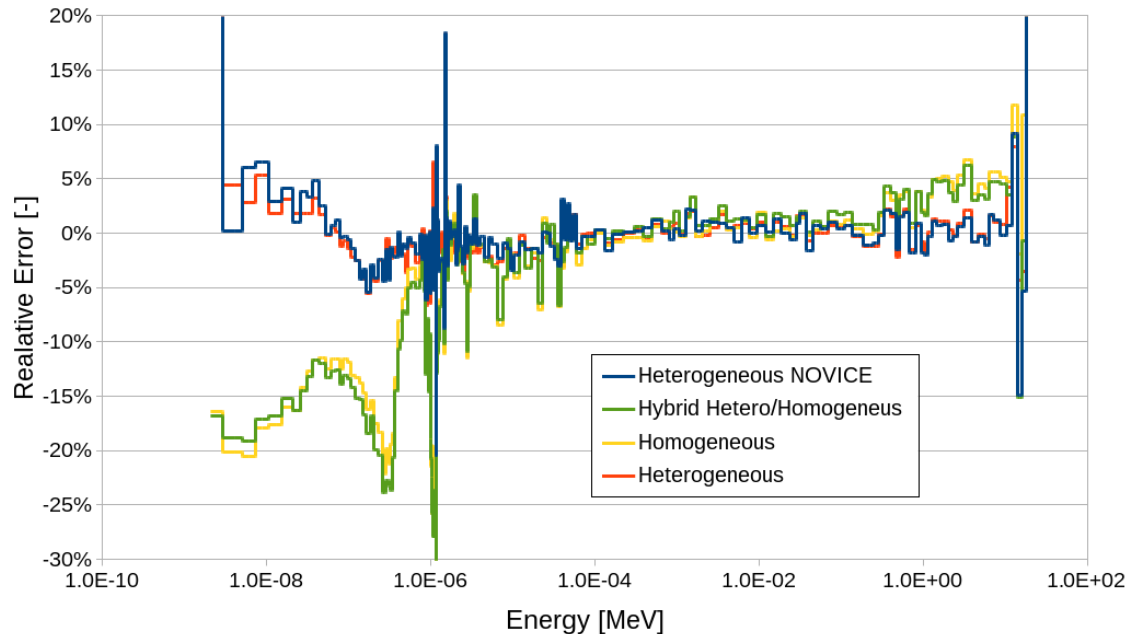
Right Assembly
(Used in Serpent Calculation)

Geometry was based on PWR:

- Pin radius = 0.3951 cm
- Pitch = 1.26 cm
- Water density $\rho \approx 0.75 \frac{g}{cm^3}$
- Boron at 1470 ppm

MONK 10A Multi-Group Spectra

Current Spectrum Relative Error



Homogeneous geometry:

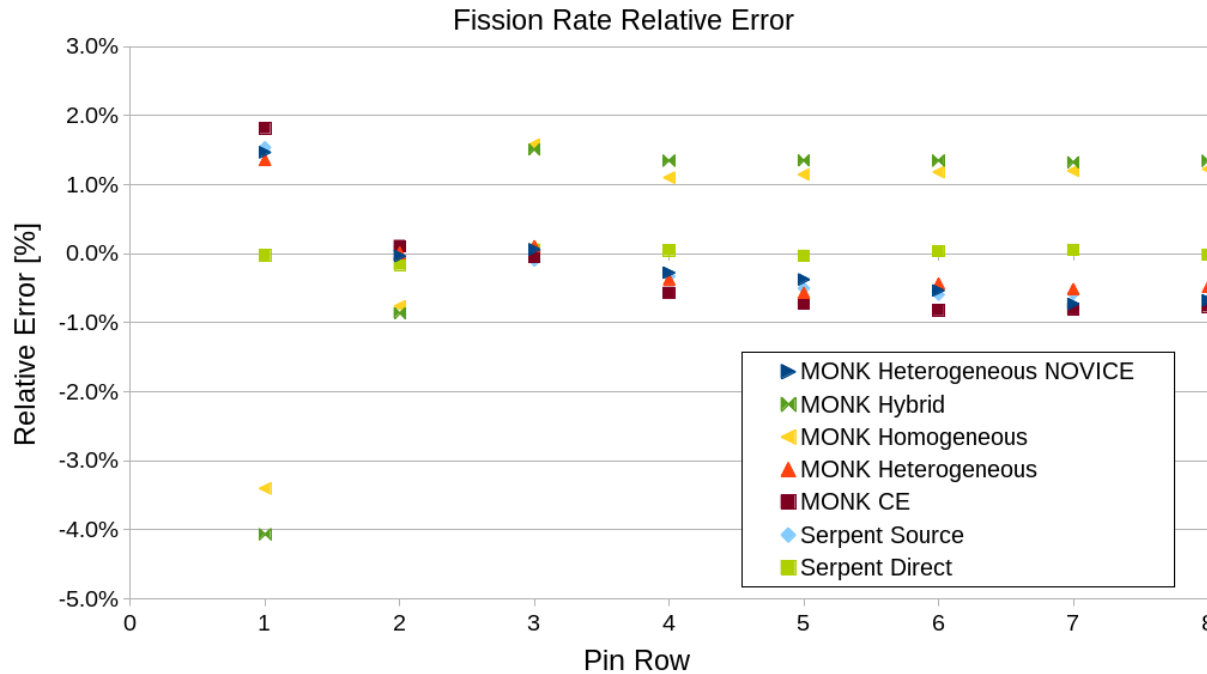
- Large Homogenisation error

Heterogeneous geometry:

- Thermal peak error
- Resonance errors
- Hardened spectrum error (criticality error)

MONK 10A NOVICE method → sub-group sampling

Fission Rate Distribution



2σ Error $\approx 0.2\%$

Normalisation to constant power

Homogenous Geometry:

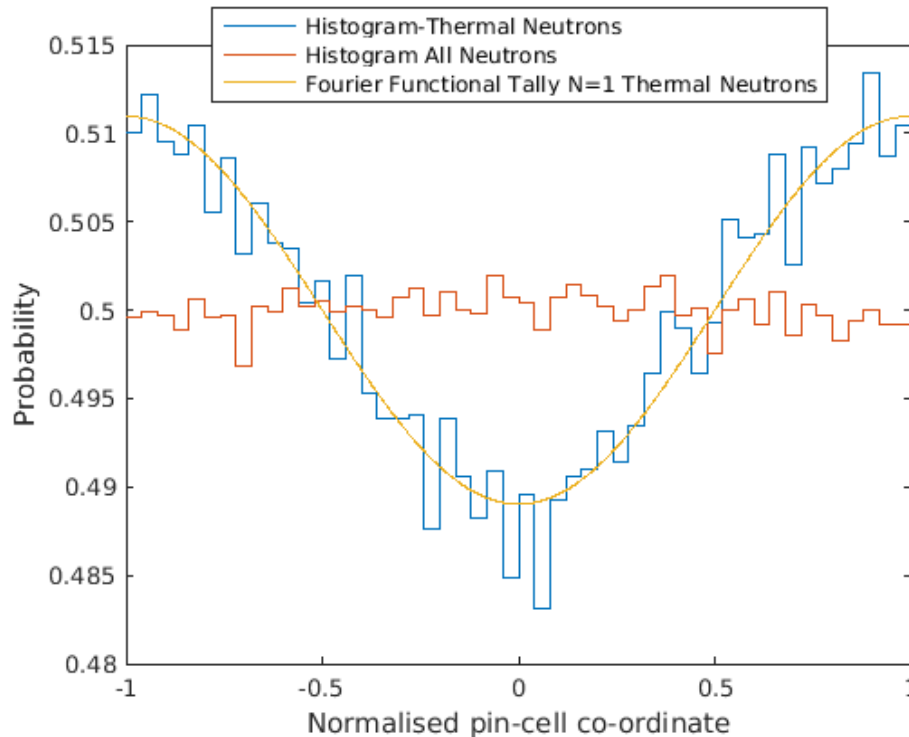
- Large Error (thermal neutrons)

Heterogenous Geometry:

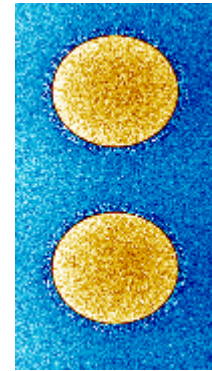
- Error in first row
- Error is larger for CE sampled spectra

Thermal pin-cell corners accumulation

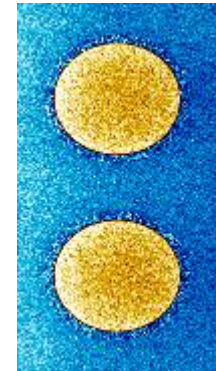
Pin-cell Neutron Distribution



CE reference



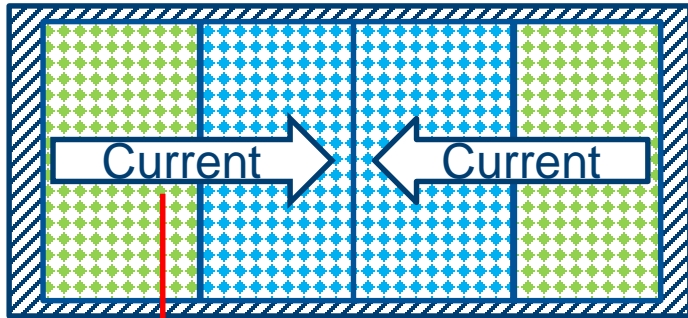
MATLAB Spectrum



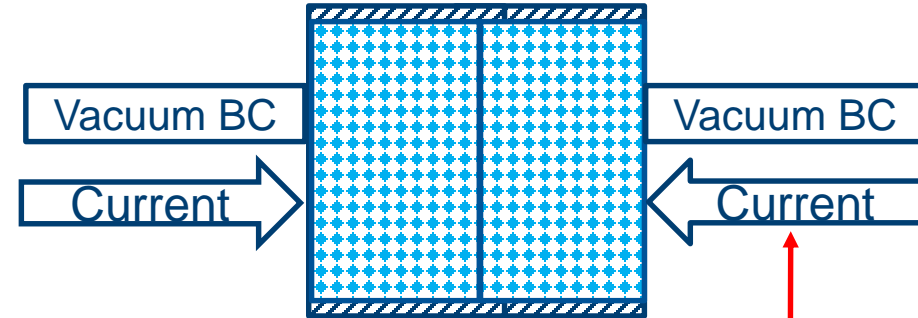
Heterogenous geometry error is caused by distribution of thermal neutrons

Isolating MG error – Source “Translation”

Serpent Eigenvalue Calculation



Serpent Fixed Source Calculation



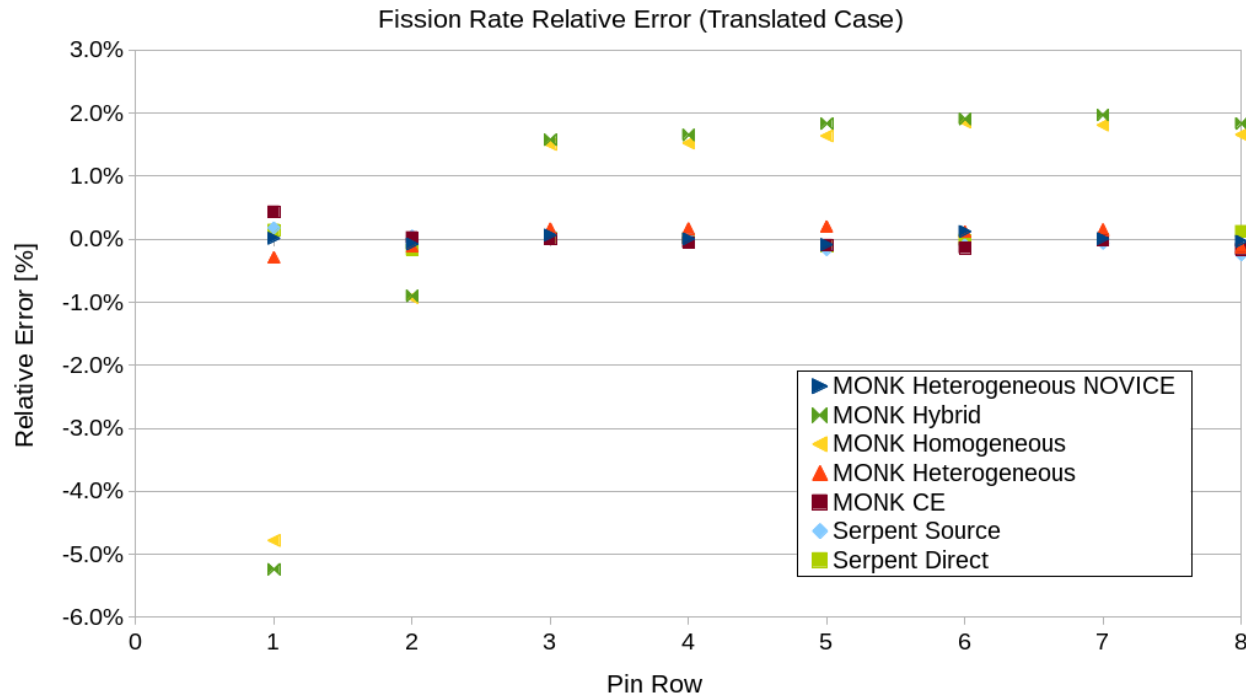
```
for every sample:  
  find energy group  
  sample new energy  
  adjust weight by MG/CE current ratio  
  write with angle/position unchanged
```

ASCII
Samples
File

MATLAB

“Translated”
Samples file

“Translated” Source Fission Rate



2σ Error \approx 0.2%

Normalisation to constant power

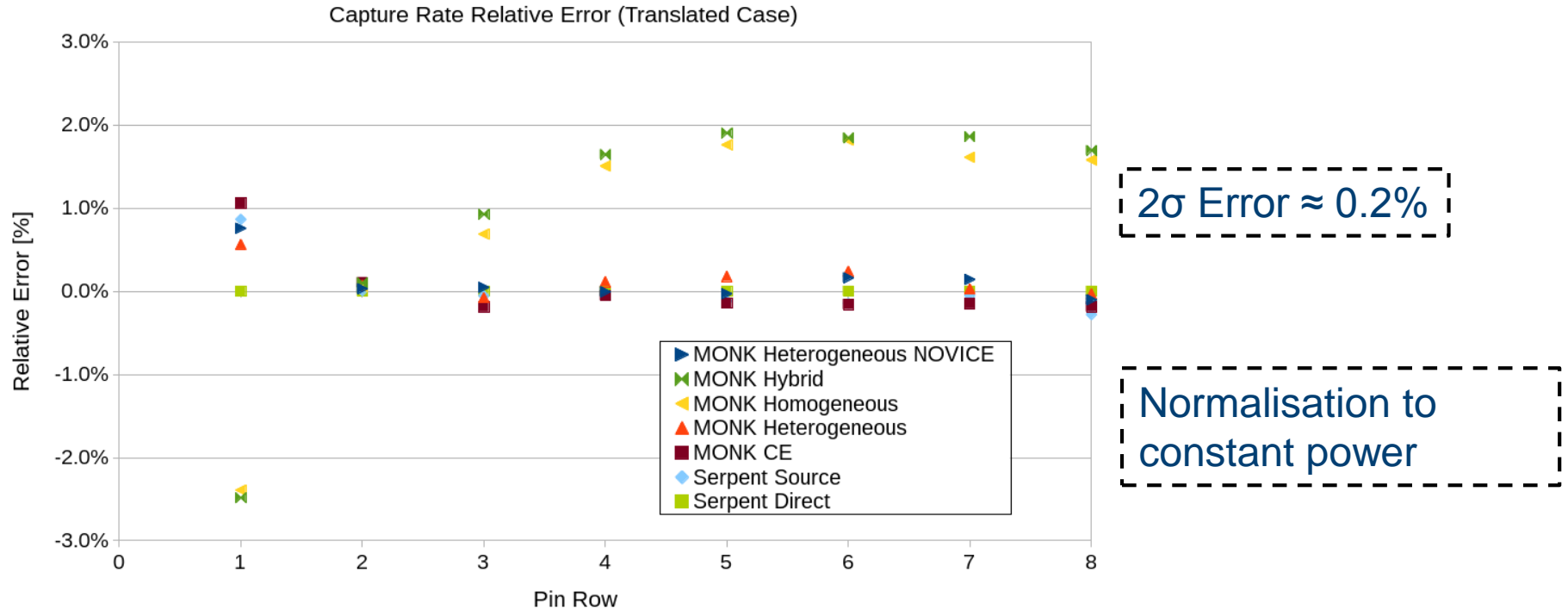
Homogeneous geometry:

- Error Increases

Heterogeneous geometry:

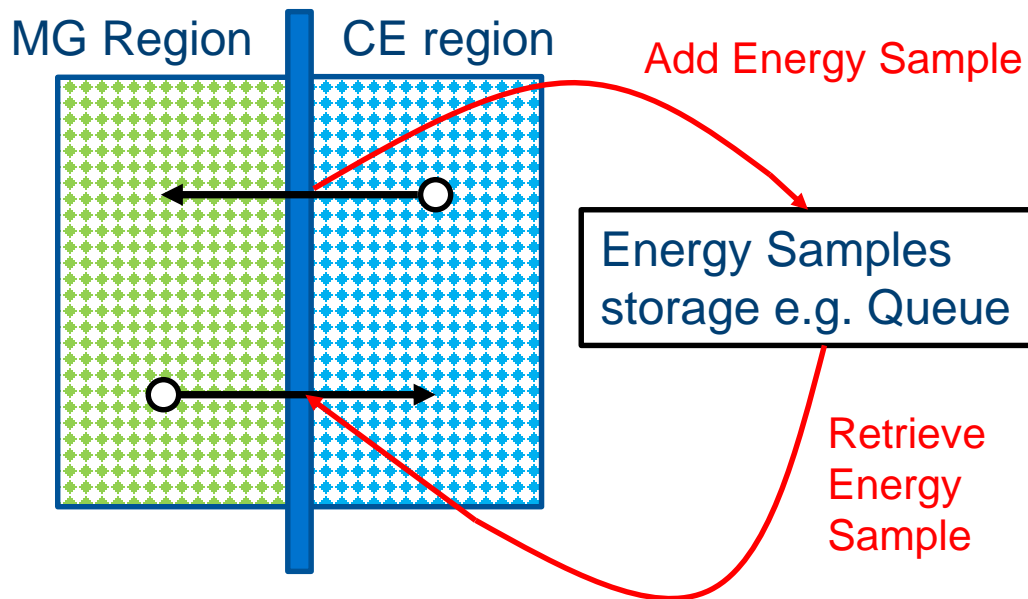
- Error is contained within 0.5%

“Translated” Source Capture Rate



No resonance flux dips \rightarrow Large capture error

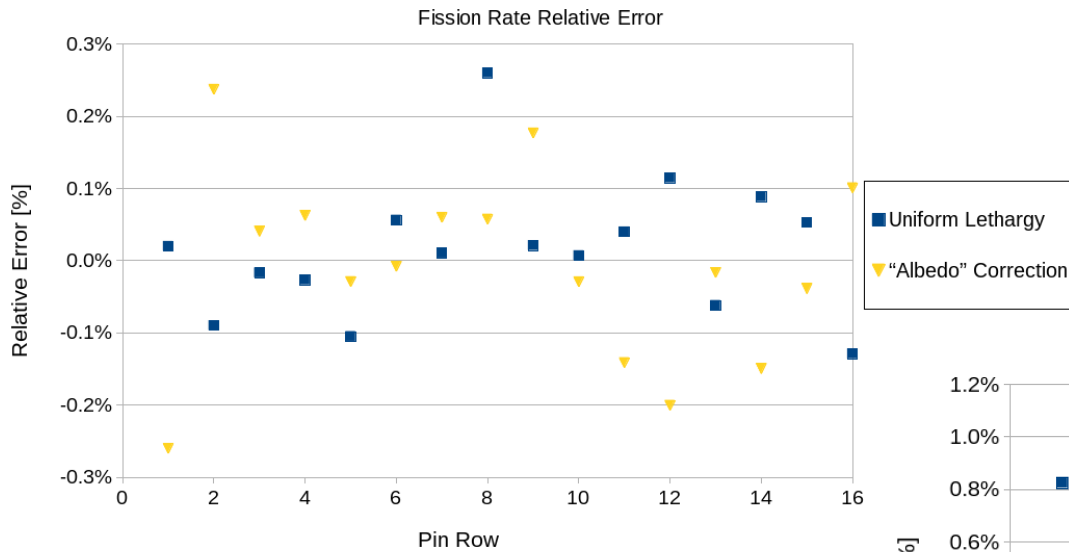
“Albedo” Correction



Approximation with “translated” sources:

1. Start with uniform distribution in lethargy
2. Run Fixed Source and get samples of leakage current
3. “Translate” Source using leakage current from previous calculation
4. Repeat steps 2-3

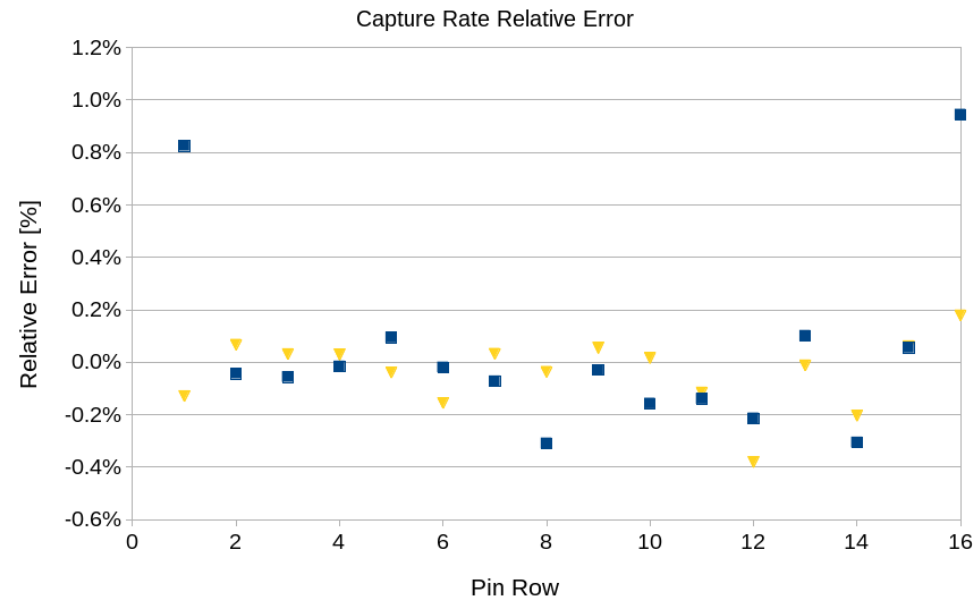
“Albedo” Correction – CE sampled MG spectrum



Spectrum was sampled in CE Serpent Reference calculation (Serpent Source case)

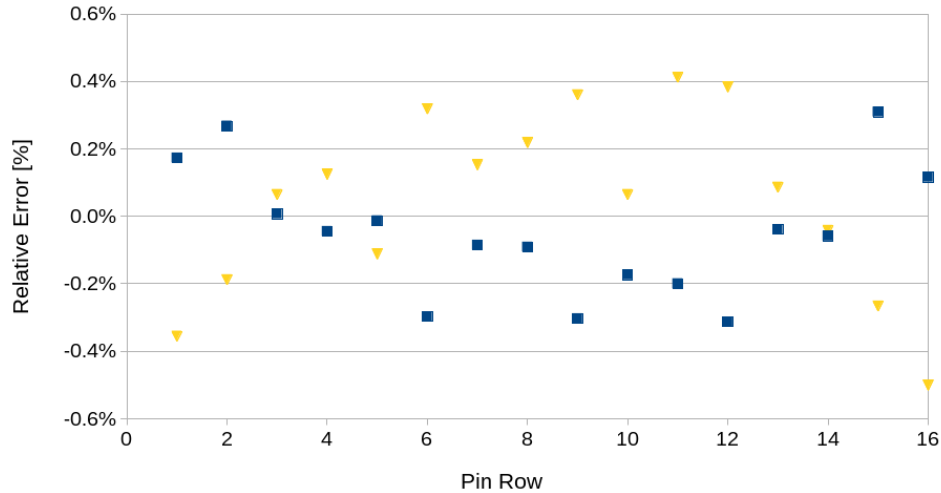
2σ Error $\approx 0.2\%$

- Significant reduction of capture error
- No statistically significant change in power



“Albedo” Correction –MG spectrum

Fission Rate Relative Error



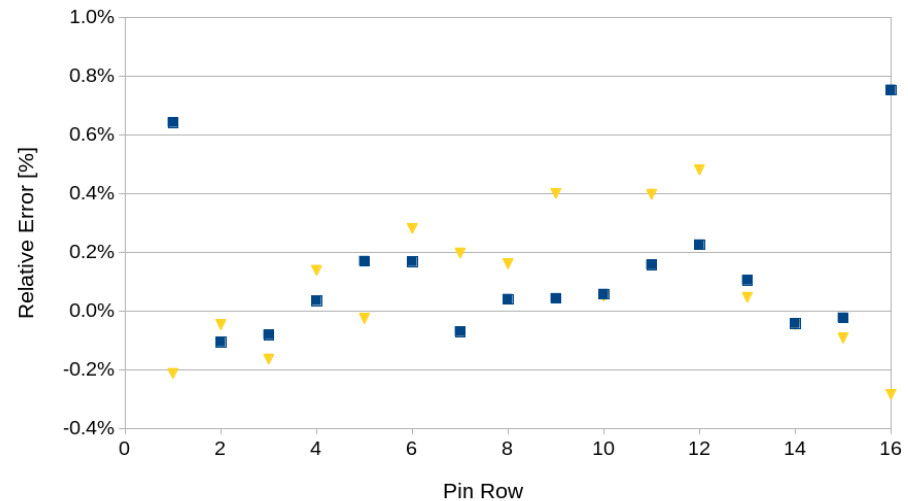
- No capture error peaking close to the boundary
- “Bowling” error in power and capture
- Likely caused by criticality error of MG spectrum

Heterogeneous MG Spectrum

■ Uniform Lethargy
▼ “Albedo” Correction

2σ Error \approx 0.2%

Capture Rate Relative Error



Conclusions

Conclusions:

- At the fidelity level boundary:
 - Homogenised to heterogeneous transition → requires correction (ADFs)
 - Spatial distribution of thermal neutrons causes large error
 - Errors due to MG representation of spectrum are small
 - “Albedo” correction → significantly improves capture accuracy
- “Variable Fidelity” will probably deliver at least 1% accuracy in power

Further work:

- Implement environment for combined MG-CE calculations
- Move to 2D and 3D problems
- Investigate using deterministic calculation as IV fidelity level

Thank you for your attention!